



RELAP5 MODELING OF A SIPHON BREAK EFFECT ON THE BRAZILIAN MULTIPURPOSE REACTOR



Tecnologia Nuclear
a Serviço da Vida

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SUMMARY

- OBJECTIVE
- BRAZILIAN MULTIPURPOSE REACTOR
- THERMAL HYDRAULIC MODEL
- LOSS OF COOLANT ACCIDENT CALCULATION ON THE RMB
- RESULTS
- CONCLUSIONS
- ACKNOWLEDGMENTS

OBJECTIVE

Simulate the Siphon Breaker phenomenon on the Core Cooling System (CCS) of the Brazilian Multipurpose Reactor (RMB) using the RELAP5/Mod3 code after of the Loss of Coolant Accident (LOCA).

BRAZILIAN MULTIPURPOSE REACTOR

RESPONSIBLE FOR THE PROJECT



CURRENT STATUS

DETAIL ENGINEERING DESIGN PHASE



INVAP



AMAZUL

RMB MAIN PURPOSES

SOCIAL APPLICATIONS

National autonomy in the production of radioisotopes for health applications, industry, agriculture and environment.

Emphasis on the Mo-99 production to ensure the technetium-99m radiopharmaceutical supply.

STRATEGIC AND INDUSTRIAL

National capacity development for testing and qualifying:

- Nuclear power reactor and naval propulsion fuels;
- New research reactor fuels and materials;

SCIENTIFIC AND TECHNOLOGICAL DEVELOPMENT

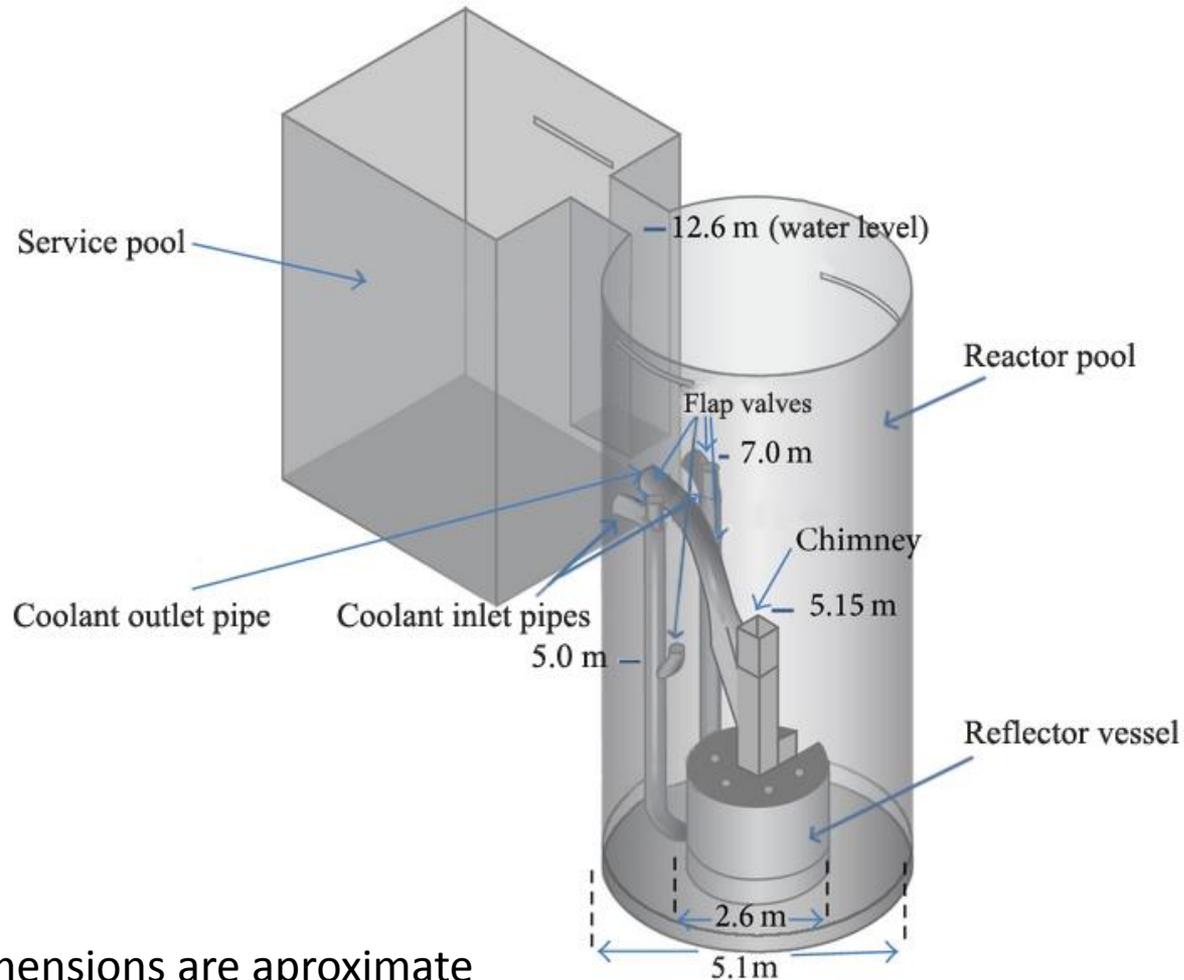
Expansion of national capacity in research and applications of nuclear techniques.

Activation Analysis Laboratory (national scientific community).

GENERAL CHARACTERISTICS OF RMB

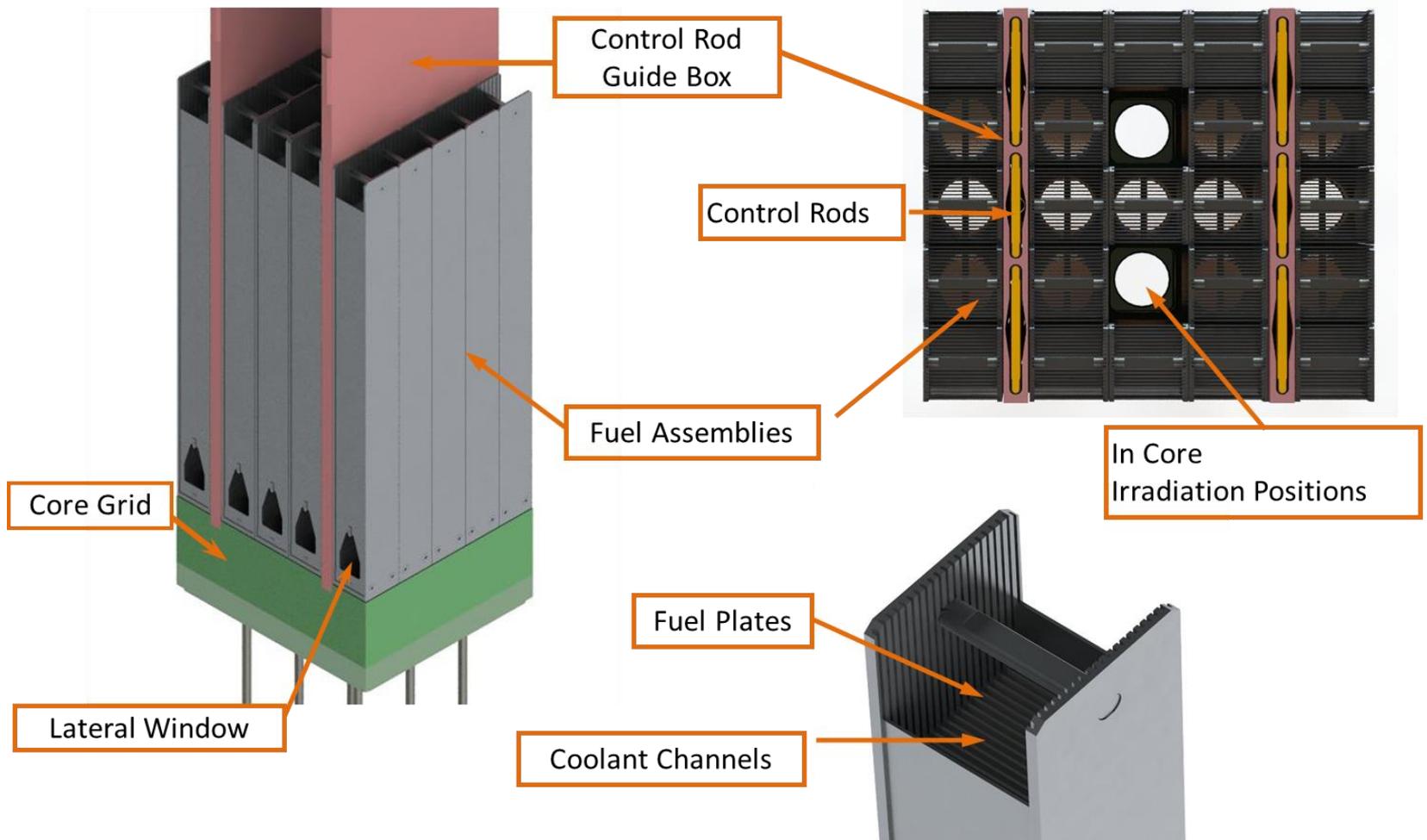
Nominal power	30 MW _{th}
Coolant / Reflector material	Light water / D ₂ O & Be
Fuel assembly type	MTR (LEU)
Core Arrangement	5 x 5 (23 Fuel Elements)
In-core irradiation position	2 positions
First Shutdown System / material	6 plates / Hafnium
Second Shutdown System	Parcial emptying of D ₂ O tank
Nuclear fuel	U ₃ Si ₂ -Al enriched at 20 %
Inlet / outlet core cooling temperature	34 °C / 41,6 °C
Core mass flow rate	3100 m ³ /h
Mass flow rate from pool to chimney	400 m ³ /h

RMB CURRENT CONCEPT

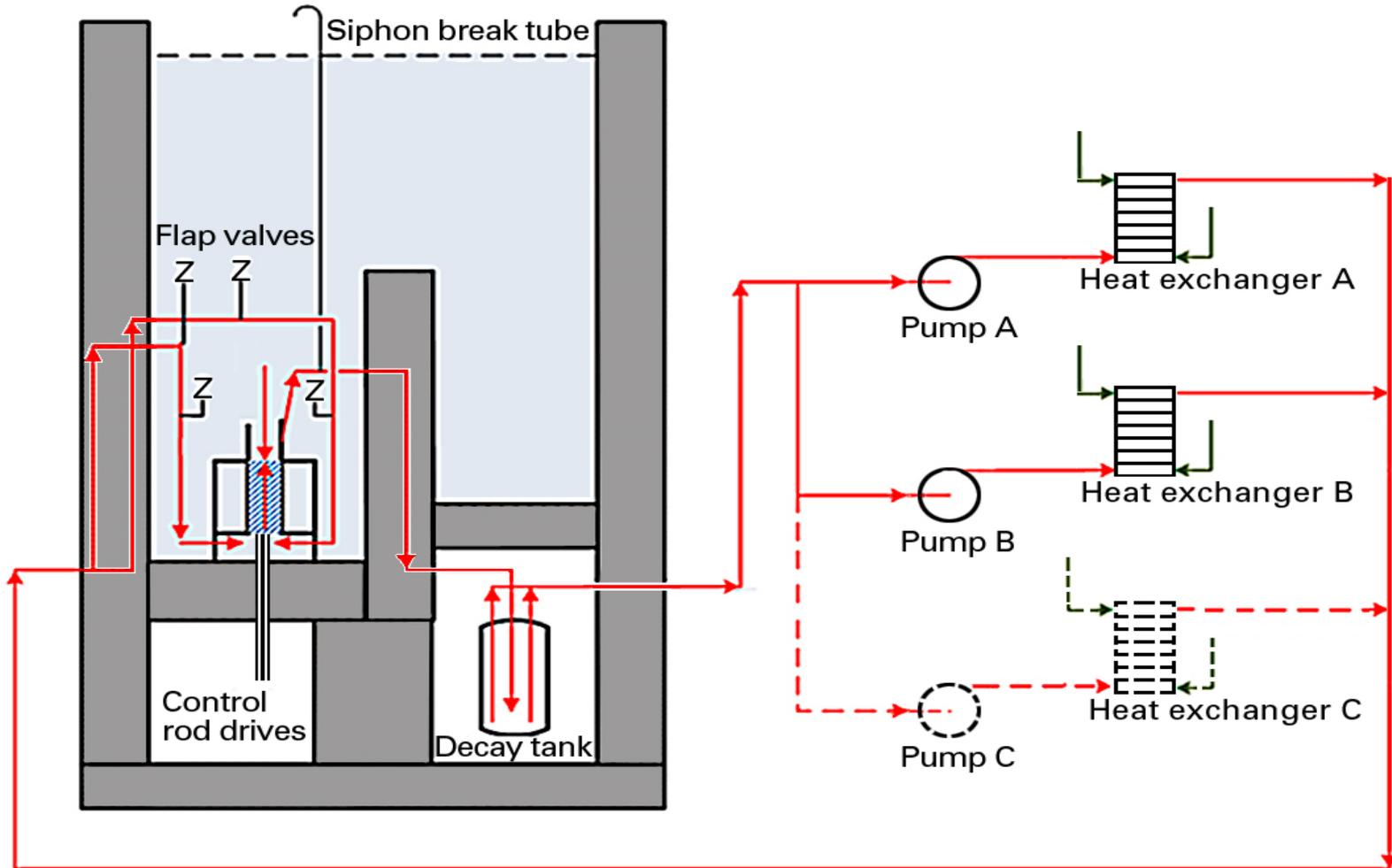


* Some dimensions are approximate

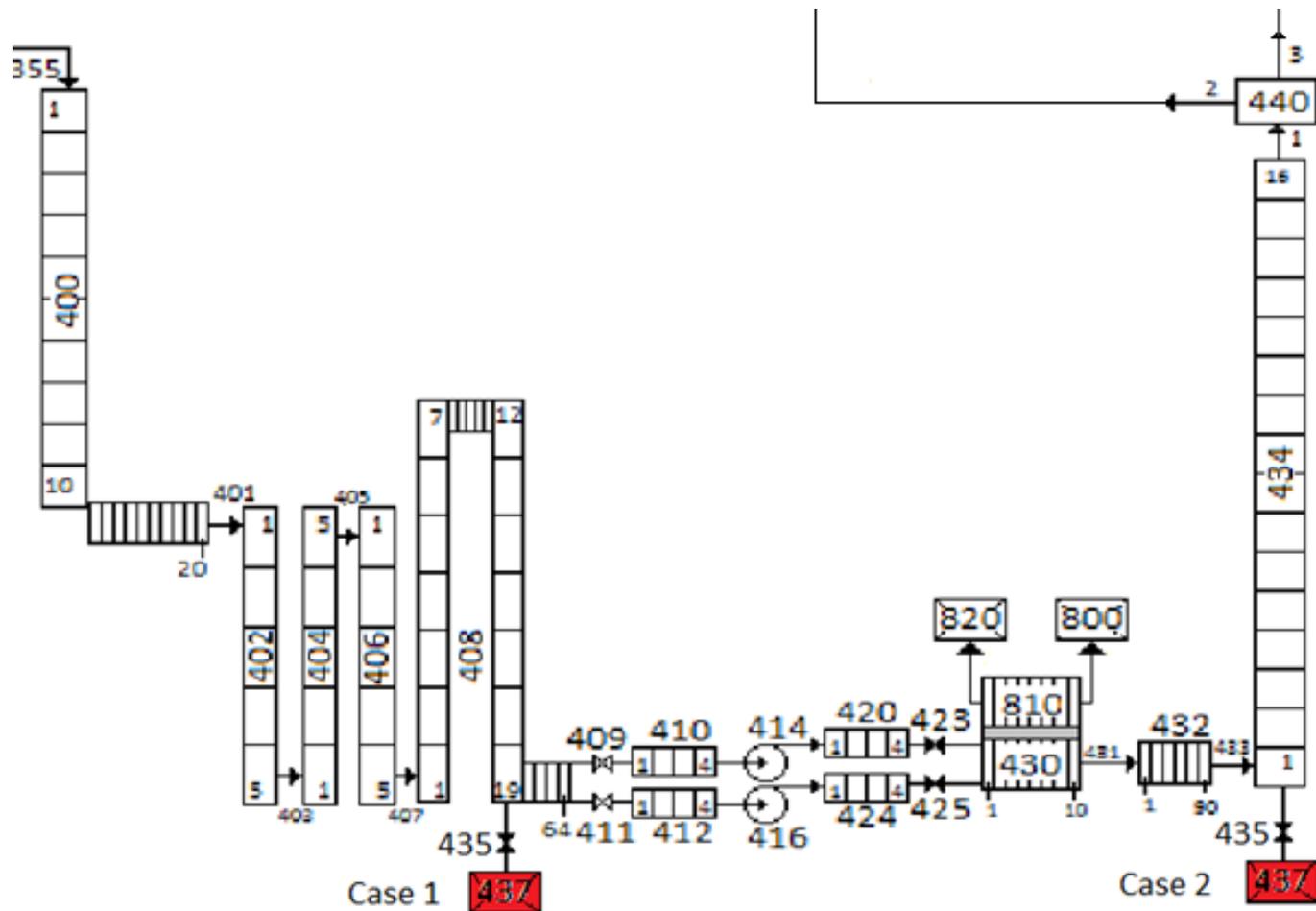
Core and Fuel Assemblies



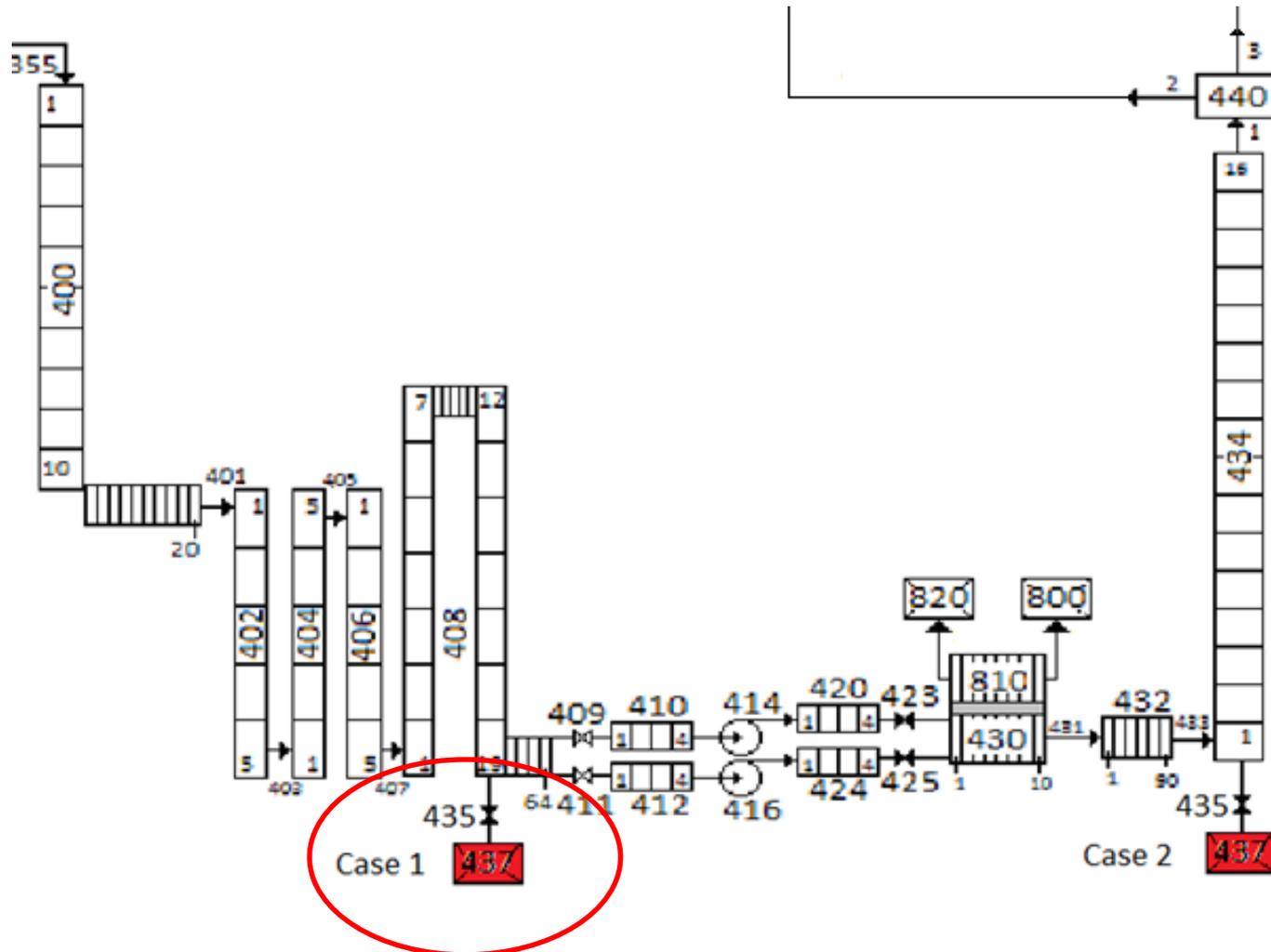
SIMPLIFIED SCHEME OF THE CCS OF THE RMB



PART OF THE RMB NODALIZATION FOR RELAP5/MOD3 HIGHLIGHTING THE POSITION OF THE LOCA EVENTS.



CASE 1: RUPTURE ON THE PUMP SUCTION LINE

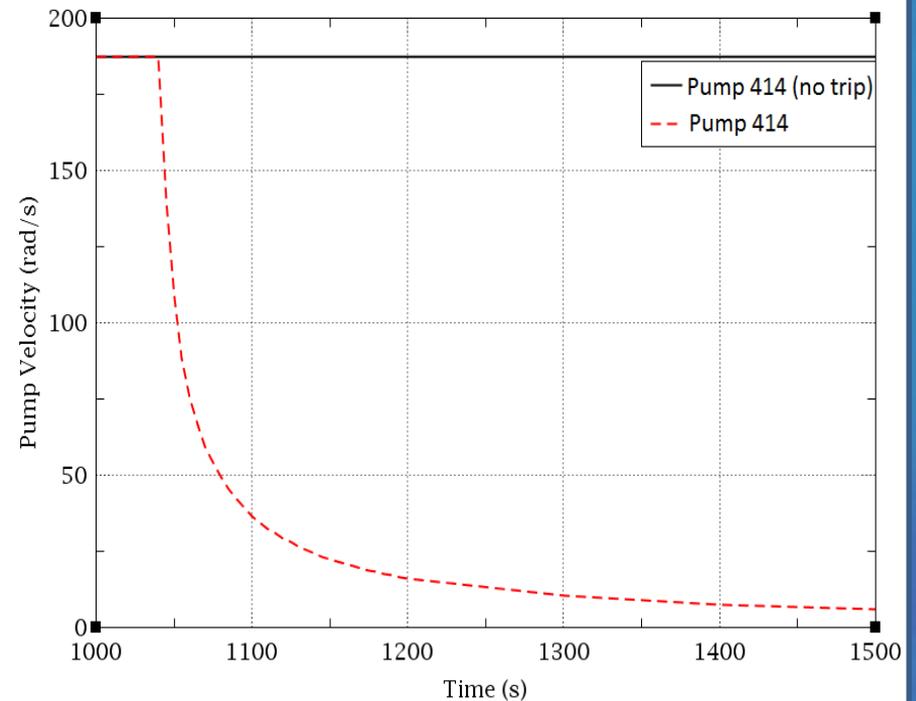
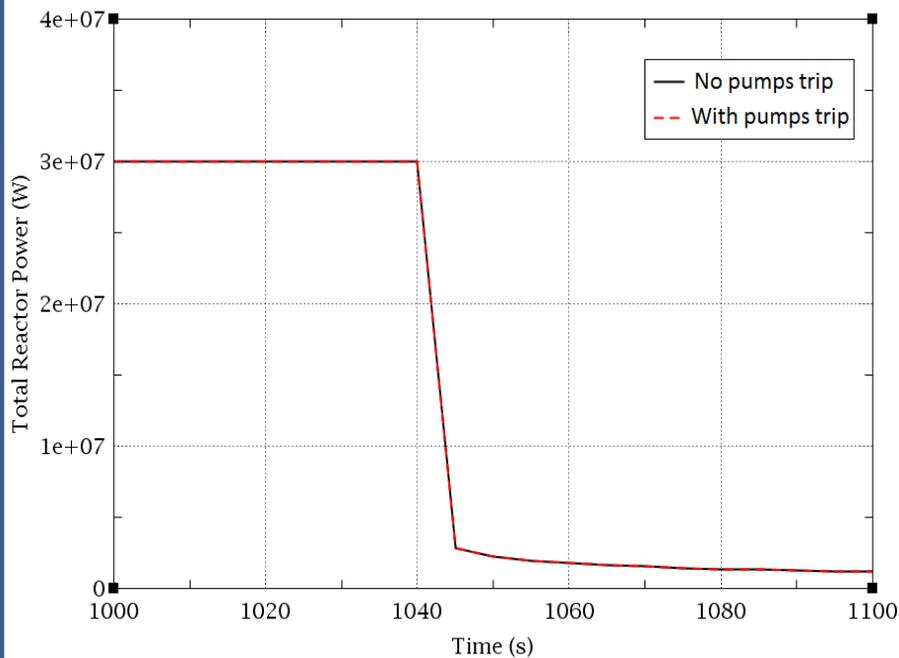


CASE 1: RUPTURE ON THE PUMP SUCTION LINE

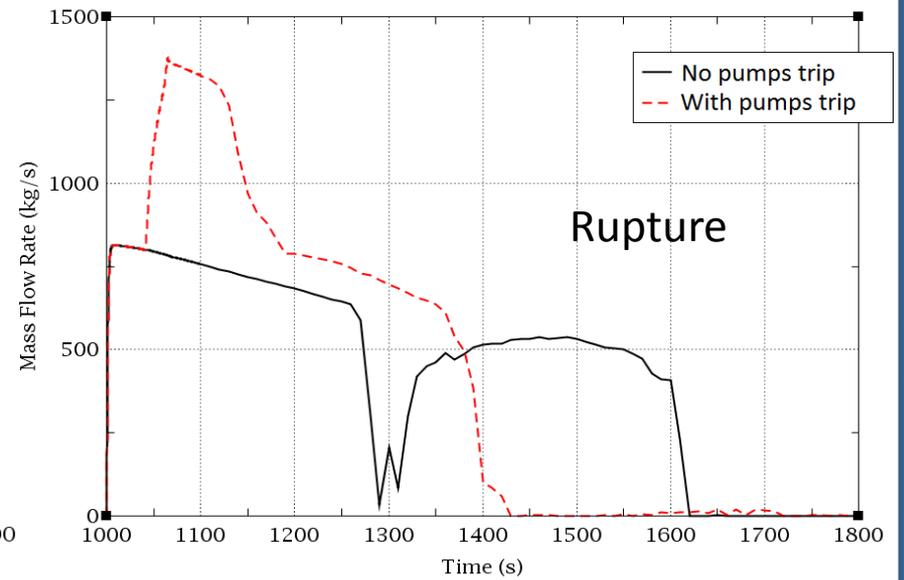
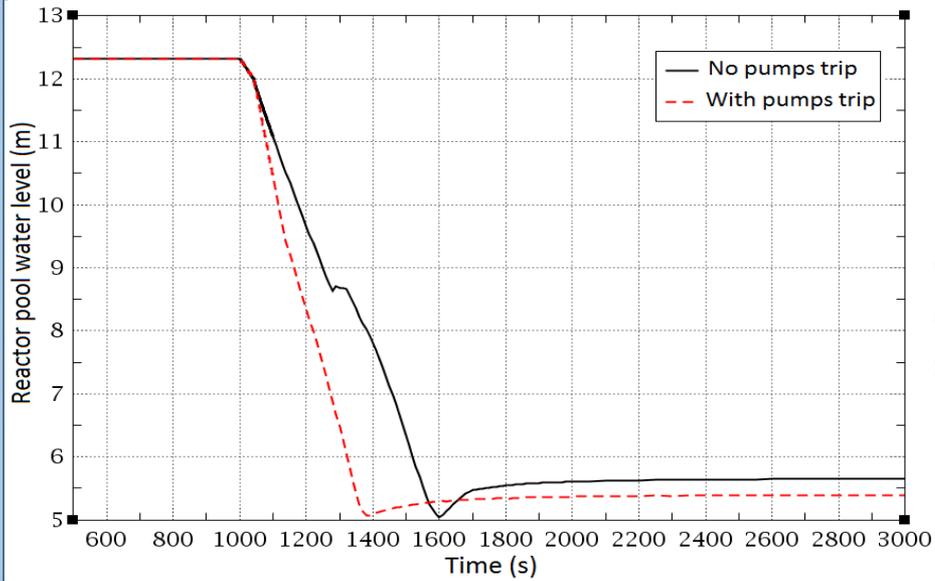
a: **No pumps trip** – pumps continue to run normally;

b: **With pumps trip** – After the reactor shutdown, a trip runs in the pump stopping it. In this event, the pump flywheels provide decreasing flow during shutdown.

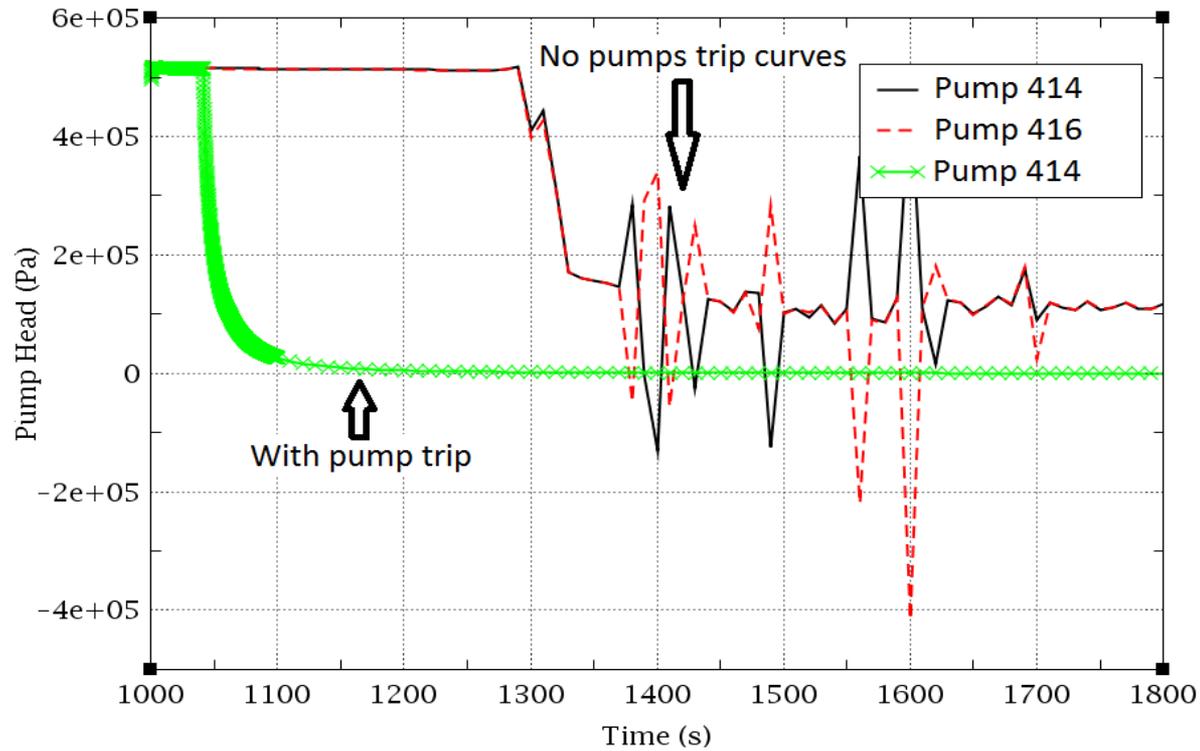
RESULTS



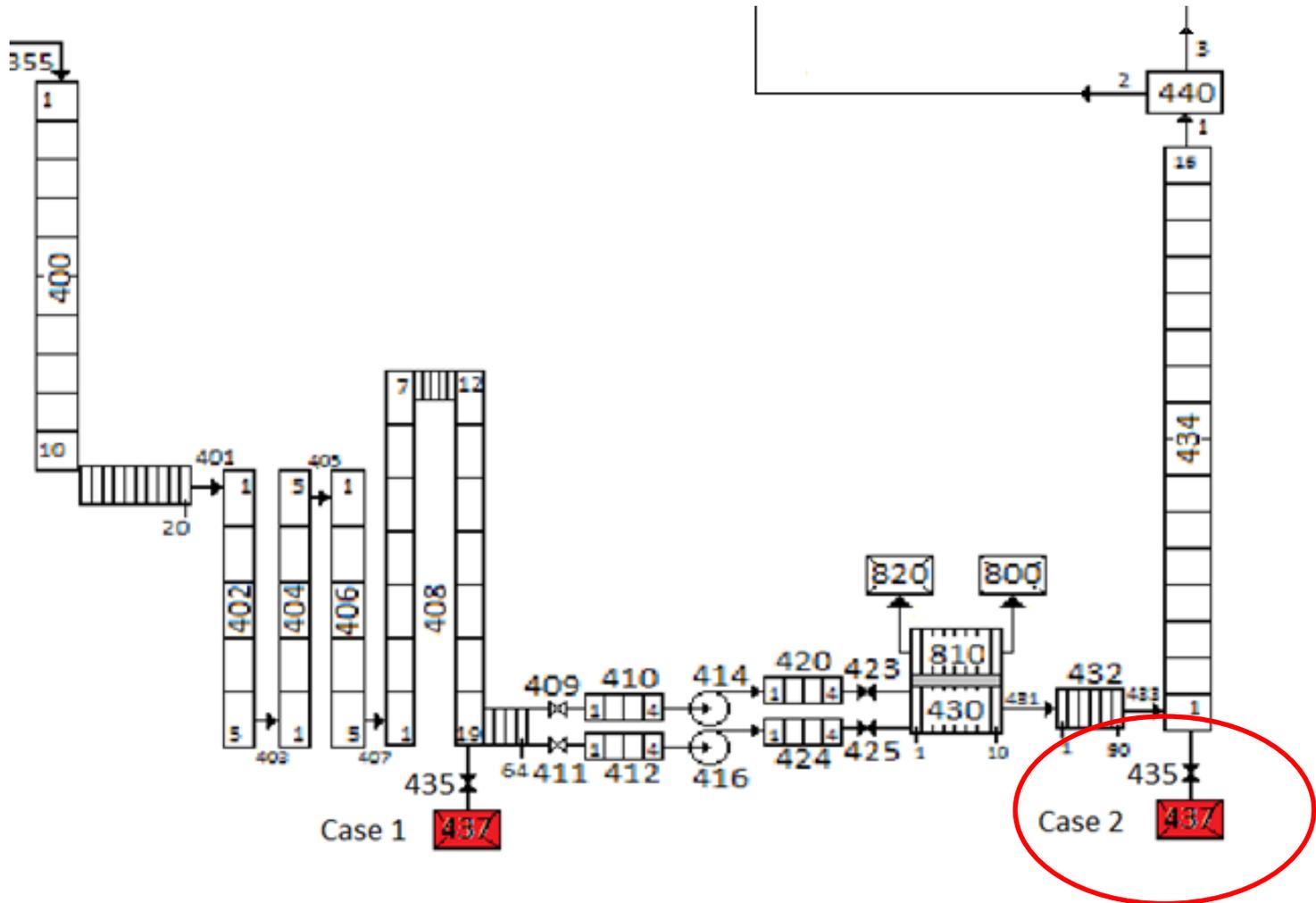
RESULTS



RESULTS



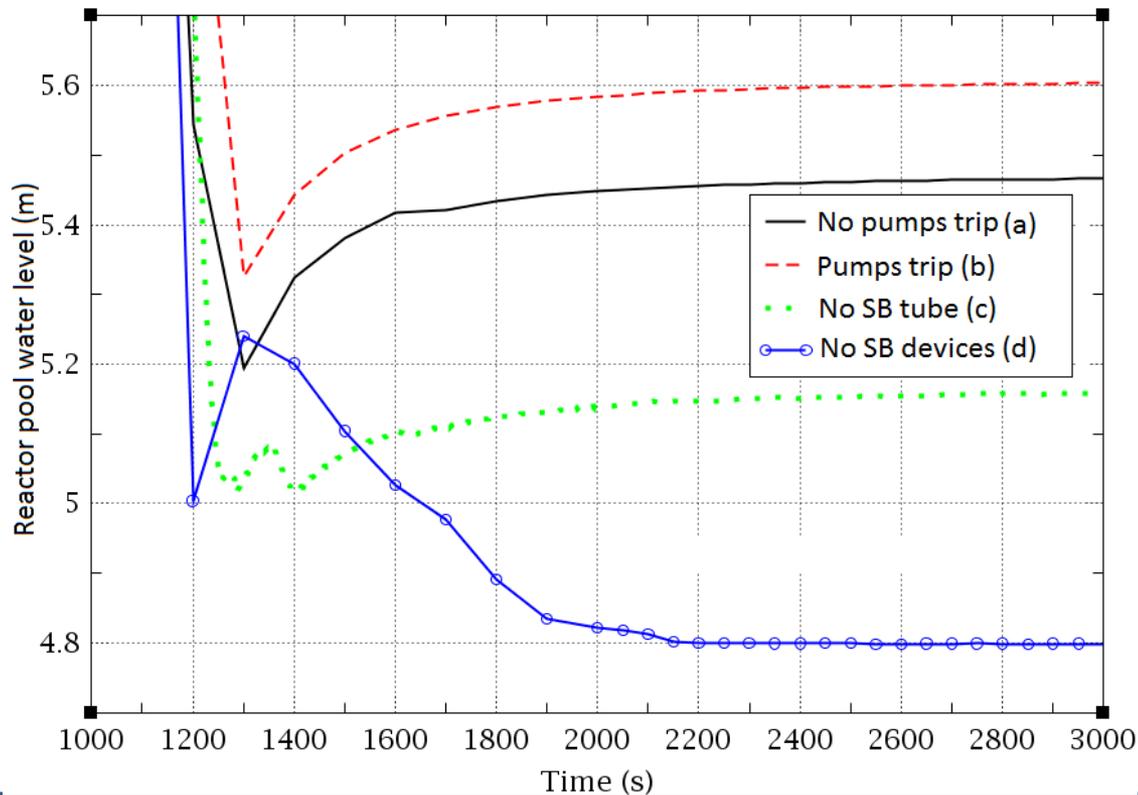
CASE 2: RUPTURE ON THE PUMP DISCHARGE LINE



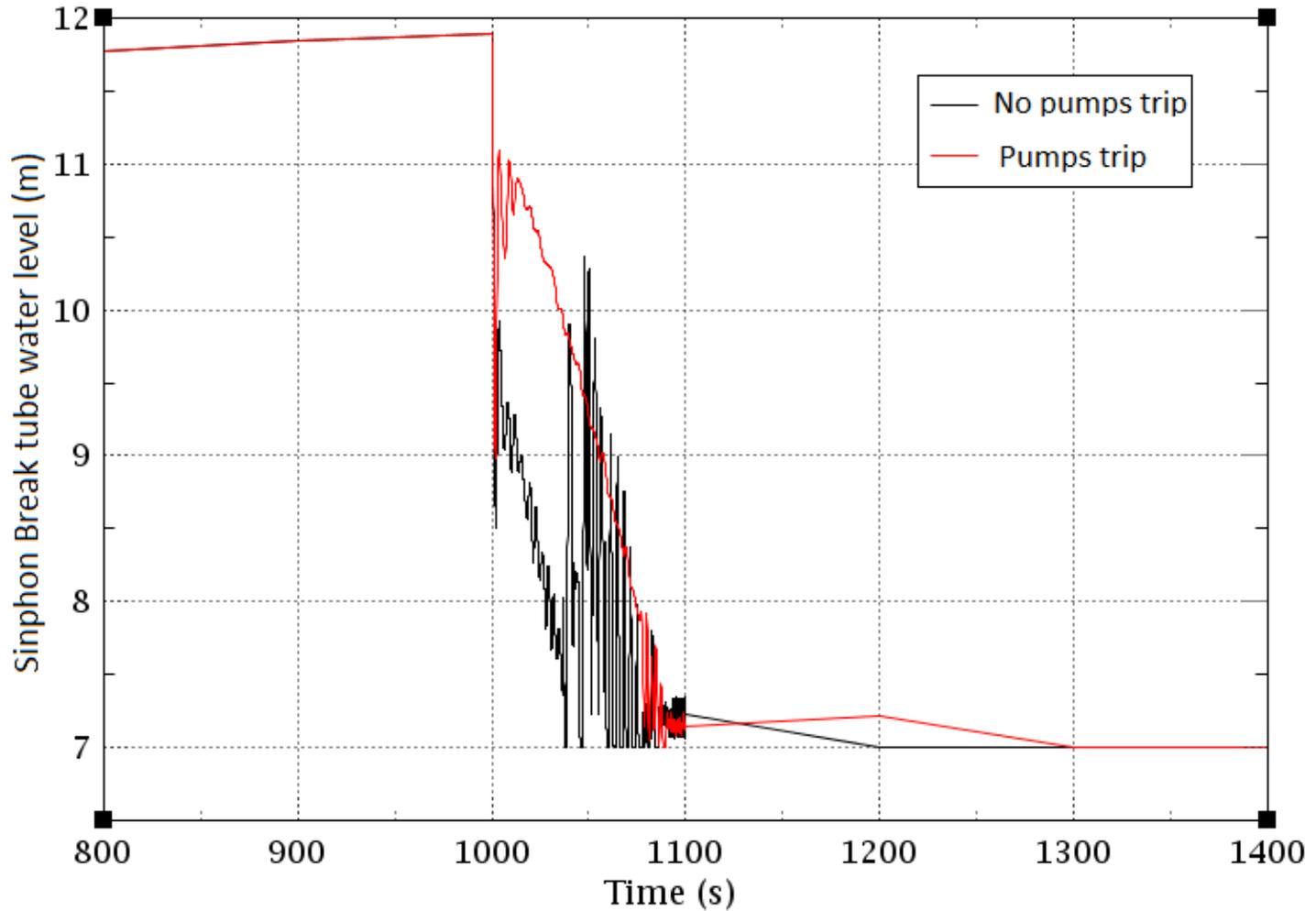
CASE 2: RUPTURE ON THE PUMP DISCHARGE LINE

- a: No pumps trip
- b: With pumps trip
- c: Pumps trip and no siphon break tube 160;
- d: Pumps trip and no siphon break devices (valve 364 and tube 160).

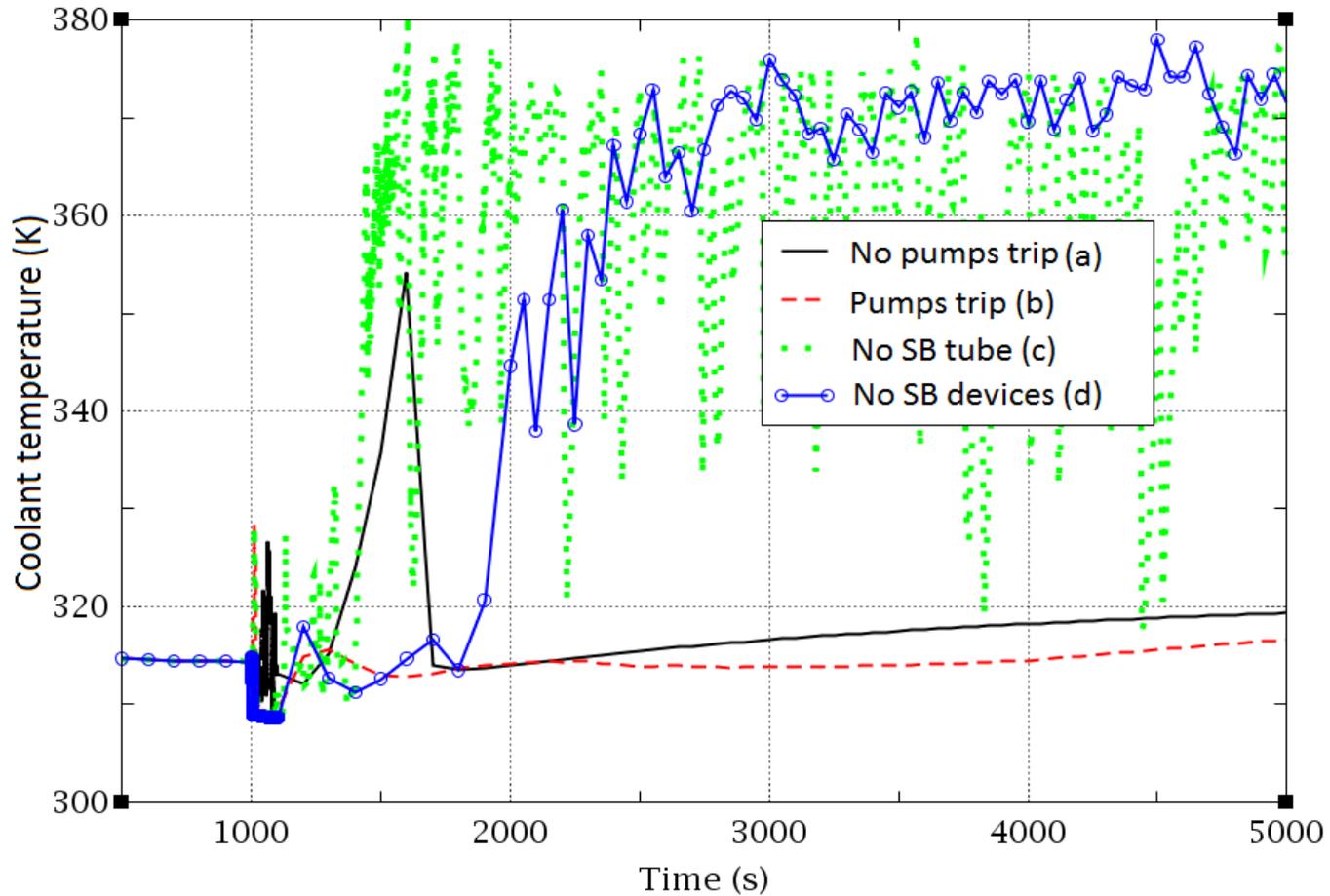
RESULTS



RESULTS



RESULTS



Core outlet temperature evolution.

CONCLUSIONS

- The phenomenological behavior of a LOCA transients in the RMB in two different positions of the Core Cooling System was analyzed with the thermal hydraulic code RELAP5/Mod3.
- The results from the numerical simulation showed that the siphon break devices are efficient to ensure the fuel physical integrity and consequently the RMB remains in a safe state during the LOCA transients.

ACKNOWLEDGMENTS

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THANK YOU FOR ATTENTION!